Radiochemical methods for the determination of plutonium, americium and curium in typical waste streams

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Abstract

Accurate and reliable analytical methods have been developed for the measurement of Pu, Am and Cm in typical waste streams from nuclear power plants. Pu, Am and Cm were separated using anion exchange and coprecipitation techniques. The solution of Am and Cm was electrodeposited and measured by α -particle spectrometry. In the case of Pu the solution was divided into two aliquots, one for liquid scintillation counting and the other for electrodeposition followed by α -particle spectrometry. To determine the chemical recovery of α emitters of the Pu fraction and the Am and Cm fraction, the sample was spiked with ²³⁶Pu and ²⁴³Am. The chemical recovery of ²⁴¹Pu was calculated using two different methods which gave reliable results. Analysis of every sample was carried out twice to examine the reproducibility of the analytical methods. The radiochemical yields of Pu and Am ranged from 80% to 100% and from 60% to 80% respectively.

1. Introduction

Characterization of the waste steams (ion exchange resins and evaporator concentrates) from nuclear power plants is needed to ascertain appropriate disposal methods. An important aspect of the characterization effort is determination of the concentrations in the waste of the major actinide elements, plutonium, americium and curium. The radionuclides of interest are α emitters, except for ²⁴¹Pu which is a β emitter. Thus suitable techniques are α -particle spectrometry (APS) to determine the concentrations of ²³⁸Pu, ^{239,240}Pu, ²⁴¹Am, ²⁴²Cm and ²⁴⁴Cm, and liquid scintillation β counting (LSC) to determine the concentration of ²⁴¹Pu.

2. Experimental method

2.1. Pretreatment of samples

We analyzed two different kinds of samples: ion exchange resins and evaporator concentrates. Firstly, in both cases, it is necessary to dissolve the samples. Dissolution of the ion exchange resins consists of attack with H_2O_2 in acid medium [1]. In the case of evaporator concentrates, the sample was attacked by a mixture of HNO₃, HCl, HClO₄ and H₂F₂.

2.2. Methods for radiochemical analysis of Pu, Am and Cm

An aliquot of the dissolved sample was taken and the tracers ²³⁶Pu and ²⁴³Am were added. The separations of Pu, Am and Cm from fission products, rare-earth and other elements have been carried out by coprecipitation and anion exchange techniques [2, 3]. The chemical procedures are shown schematically in Fig. 1.

After anion exchange purification the solution of Am and Cm was electrodeposited followed by APS. The solution of Pu was divided into two aliquots, one for LSC and the other for electrodeposition followed by APS.

2.3. Determination of α active radionuclides

The solution of Pu or Am and Cm was electrodeposited onto a stainless steel disc, following the method of Hallstadius [4]. The α spectra were taken with 450 mm² silicon implanted detectors.

2.4. Determination of ²⁴¹Pu

The purified Pu fraction was transferred to a 22 ml glass liquid vial and 10 ml of counting cocktail (Instagel) was added. β and α particle counting was done by liquid scintillation using the ranges 0–19 keV and 135–340 keV respectively. The efficiency of liquid scintillation β counting for ²⁴¹Pu is similar to that for ³H (maximum 60%) and the efficiency of the liquid scintillation α counting is practically 100% [5].

The activity of 241 Pu in the sample C_{41} is given by the following expression:

$$C_{41} = \frac{A_{41}}{R_{\rm s}MF}$$

where M is the amount of sample (g or ml) analyzed, F is the fraction of the sample solution used for LSC,

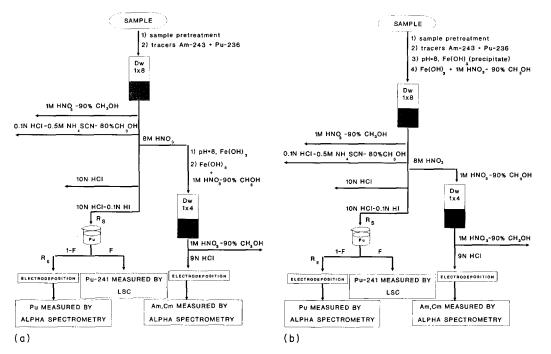


Fig. 1. Separation scheme of Pu, Am and Cm for ion exchange resin (a) and evaporator concentrate (b) samples.

Reference	²³⁸ Pu	^{239, 240} Pu	²⁴¹ Pu (Method A)	²⁴¹ Pu (Method B)
 R-1	44±1	7.9±0.2	869±54	776±42
R-1	45 ± 2	8 ± 1	847 ± 94	758±54
R-2	86 ± 5	46 ± 3	2388 ± 143	2399 ± 96
R-2	89 ± 5	48 ± 3	2550 ± 204	2348 ± 141
R-3	91 ± 7	48 ± 2	2806 ± 224	2600 ± 156
R-3	98 ± 8	53 ± 5	2700 ± 162	2800 ± 280
C-1	0.26 ± 0.02	0.19 ± 0.01	16 ± 4	17 ± 3
C-1	0.23 ± 0.02	0.18 ± 0.02	15 ± 2	14 ± 1
C-2	0.215 ± 0.004	0.083 ± 0.004	34 ± 3	36 ± 3
C-2	0.219 ± 0.004	0.079 ± 0.003	35 ± 4	37 ± 4
C-3	0.14 ± 0.01	0.070 ± 0.008	10.0 ± 0.06	9.0 ± 0.5
C-3	0.16 ± 0.02	0.086 ± 0.01	9.7 ± 0.04	9.5 ± 0.4

TABLE 1. Results of the plutonium analysis of ion exchange resins and evaporator concentrates

R, ion exchange resin sample (Bq $g^{-1}\pm 2\sigma$); C, concentrate evaporator sample (Bq $ml^{-1}\pm 2\sigma$).

 A_{41} is the activity of ²⁴¹Pu (Bq) measured by LSC, and R_s is the chemical separation yield of ²⁴¹Pu which can be calculated using two different methods.

2.4.1. Method A

 $R_{\rm s}$ of ²⁴¹Pu is the same as that of ²³⁶Pu before electrodeposition:

$$R_{\rm s} = \frac{R_{\rm T}}{R_{\rm E}(1-F)}$$

where (1-F) is the fraction of sample solution used for APS, $R_{\rm T}$ is the total recovery of ²³⁶Pu that can be calculated by APS, and $R_{\rm E}$ is the electrodeposition recovery,

$$R_{\rm E} = \frac{A_{\rm 6D}}{A_{\rm 6D} + A_{\rm 6N}}$$

where A_{6D} is the activity of electrodeposited ²³⁶Pu, calculated by APS, and A_{6N} is the activity of ²³⁶Pu which is not electrodeposited. To determine this activity it is necessary to do a second electrodeposition with the solution left after the first electrodeposition and an added known amount of a tracer. The best tracer would be ²⁴²Pu but it was not available in our laboratory. We had a standard ²³⁸Pu sample which could be used as tracer. This tracer has the advantage that its halflife and its immediate daughter product ²³⁴U are both long lived, thus once purified, the tracer remains substantially pure for a long time.

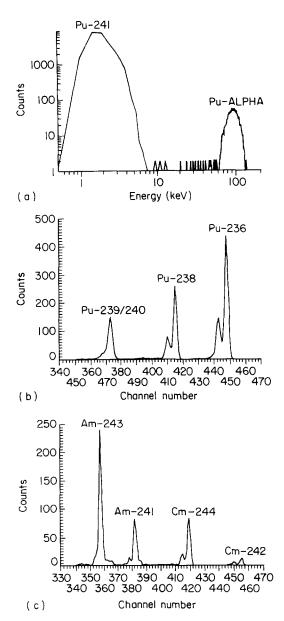


Fig. 2. Liquid scintillation β counting spectrum of Pu (a), α spectrum of Pu (b) and α spectrum of Am and Cm (c).

In each source, the activity ratio 238 Pu/ 236 Pu is calculated by APS then A_{6N} is

$$A_{6N} = \frac{T_8}{R_{86}^{\rm T} - R_{86}}$$

where T_8 is the activity of ²³⁸Pu (Bq) added to the solution, R_{86}^{T} is the activity ratio ²³⁸Pu/²³⁶Pu for the second source, and R_{86} is the activity ratio ²³⁸Pu/²³⁶Pu for the first source.

2.4.2. Method B

Another method to calculate R_s is measurement of the relations ²³⁸Pu/²³⁶Pu and ^{239,240}Pu/²³⁶Pu by APS using the total α activity of Pu isotopes found by LSC:

TABLE 2. Results of the americium and curium analysis of ion exchange resins and evaporator concentrates

Reference	²⁴¹ Am	²⁴² Cm	²⁴⁴ Cm
R-1	6.2 ± 0.4	81±4	33±1
R-1	5.6 ± 0.2	80 ± 4	32 ± 1
R-2	10.3 ± 0.6	56 ± 3	28 ± 2
R-2	11.6 ± 0.9	59±5	29 ± 2
R-3	12.6 ± 0.8	58 ± 3	30 ± 2
R-3	11.3 ± 0.7	55 ± 3	28 ± 2
C-1	0.076 ± 0.001	0.0018 ± 0.0002	0.024 ± 0.002
C-1	0.080 ± 0.007	0.0017 ± 0.0004	0.025 ± 0.004
C-2	0.057 ± 0.002	0.230 ± 0.005	0.161 ± 0.003
C-2	0.054 ± 0.002	0.220 ± 0.004	0.160 ± 0.003
C-3	0.040 ± 0.002	0.0005 ± 0.0001	0.013 ± 0.001
C-3	0.043 ± 0.004	0.0007 ± 0.0002	0.012 ± 0.001

R, ion exchange resin sample (Bq $g^{-1}\pm 2\sigma$); C, concentrate evaporator sample (Bq $ml^{-1}\pm 2\sigma$).

$$R_{\rm s} = \frac{A_{\rm T}}{(1 - R_{\rm 86} + R_{\rm 96})T_6F}$$

where F is the fraction of the sample solution used for LSC, and $A_{\rm T}$ is the total α activity (Bq) in the fraction F determined by LSC, T_6 is the activity of ²³⁶Pu (Bq) added to the sample, R_{86} is the activity ratio ²³⁸Pu/²³⁶Pu determined by APS, and R_{96} is the activity ratio ^{239,240}Pu/²³⁶Pu determined by APS.

3. Discussion

The procedures described for the separation of Pu, Am and Cm are efficient methods. This is indicated by the liquid scintillation counting spectrum of Pu and the good resolution of the peaks of Pu, Am and Cm in the α -ray spectra (Fig. 2). The results of the analysis are given in Tables 1 and 2. To examine the reproducibility of the present method, the analyses were carried out twice. The radiochemical yields of Pu and Am ranged from 80% to 100% and from 60% to 80% respectively.

The results of the analysis of ²⁴¹Pu obtained by methods A and B are given in Table 1. Experience of both methods has shown that the results are generally reliable.

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